

## ASME Standards & Certification Energy-Related Products and Services

### Nuclear Energy

#### Codes and Standards

|   |   |
|---|---|
| Atmospheric Water Cooling Equipment (PTC 23)  | Nuclear Air and Gas Treatment (AG-1)  |
| Condensers, Steam, Surface (PTC 12.2)   | Nuclear Air Treatment Systems Testing (N510)  |
| Construction of Nuclear Facility Components (BPVC Section III Division 1)   | Nuclear Code Cases (Sections III and XI)  |
| Construction of Concrete Containments (BPVC Section III Division 2)   | Nuclear Power Plant Air-Cleaning Units and Components (N509)  |
| Construction of Containments for Transportation & Storage of Spent Nuclear Fuel and High Level Radioactive Material and Waste (BPVC Section III Division 3) | Nuclear Power Plants, Operation and Maintenance (OM)  |
| Construction of Cranes, Monorails, and Hoists for Use in Nuclear Facilities (NUM-1)   | Power Plants, Performance-Related Outage Inspections (POM 101)  |
| Construction of Overhead and Gantry Cranes for Use in Nuclear Facilities (NOG-1)  | Power Plants, Steam, Performance Monitoring Guidelines (PTC PM)   |
| Feedwater Heaters, Closed (PTC 12.1)  | Reheaters, Moisture Separator (PTC 12.4)  |
| Heat Exchangers, Air-Cooled (PTC 30)  | Repair of Pressure Equipment and Piping (PCC-2)   |
| Heat Exchangers, Single Phase (PTC 12.5)  | Qualification of Active Mechanical Equipment Used in Nuclear Power Plants (QME-1)                                   |
| High Purity Water Treatment Systems (PTC 31)  | Quality Assurance Requirements for Nuclear Facility Applications (NQA-1)  |
| In-Service Inspection of Nuclear Power Plant Components (BPVC Section XI)   | Steam and Water Sampling, Conditioning, and Analysis in the Power Cycle (PTC 19.11)                                 |
| In-Service Testing of Nuclear Air Treatment, Heating, Ventilating, and Air-Conditioning Systems (N511)  | Turbines, Steam (PTC 6)   |
| Inspection Planning Using Risk-Based Methods (PCC-3)  | Turbines, Steam, Prevention of Water Damage (TDP-1)   |
| Large/Early Release Frequency Probabilistic Risk Assessment for Nuclear Power Plant Applications Equipment used in Nuclear Power Plants (RA-S)              | Verification and Validation in Computational Integrated System Thermal Fluids Behavior (V&V-30) (under development) |

#### Training Courses

|   |
|---|
| Details in Design - Live  |
| Developing an In-Service Testing Program - Live   Online                                    |
| Fabrication & Details in Fabrication – Live   |
| HDPE Piping - Live   Online   |
| In-Services Testing for Pumps - Live   Online   |
| In-Services Testing for Valves - Live   Online  |
| Materials and Details in Materials - Live   |
| Overview of ASME Codes for the Nuclear Industry - Live   Online                             |
| Overview of ASME BPV Section III - Live   Online  |
| Overview of Design Documents and Relationship to ASME Code – Live                           |
| Power Plant Construction – Online (under development)                                       |
| Probabilistic Risk Assessment Overview - Online   |
| Probabilistic Risk Assessment Training - Live   |
| QA Considerations for New Nuclear Facility Construction - Live   Online (under development) |
| Risk-Informed In-Service Testing Program - Live   Online                                    |
| Section III, Division 1, Class 1 - Live   Online  |
| Section III, Division 1, Class 2 & 3 - Live   Online  |
| Section XI: In-service Inspection of Nuclear Power Plant Components - Live                  |
| Nuclear Certificate Program   |
| NQA-1 Requirements for Computer Software Used in Nuclear Facilities                         |

Note: Other ASME standards may be applicable.

For more information visit <http://catalog.asme.org/> or call 1-800-THE-ASME (800-843-2763).

## ASME Standards & Certification Energy-Related Products and Services

### *Nuclear Energy*

#### Technical Reports and Guides

Analysis of Selected Non Destructive Examination (NDE) Methodologies for the Assessment of Cracking in Concrete Containments (STP-NU-069)  
ASME Code Considerations for the Intermediate Heat Exchanger (IHX) (STP-NU-038)  
Bolted Flange Joint Assemblies, Guidelines for Pressure Boundary (PCC-1)  
Code Comparison Report for Class 1 Nuclear Power Plant Components (STP-NU-051)  
Code Development Roadmap for HDPE Pipe in Nuclear Service (STP-NU-057)  
Comprehensive Comparison of International Quality Standards (STP-NU-062)  
Conservatism in the B2 And B2' Index (STP-NU-008)  
Correct and Extend Allowable Stress Values for 304 and 316 Stainless Steel (STP-NU-063)  
Corrections to Stainless Steel Allowable Stresses (STP-NU-059)  
Corrosion of A193 Grade B7 Bolt Material in BWR Sodium Pentaborate Solutions (STP-NU-068)  
Creep and Creep-Fatigue Crack Growth at Structural Discontinuities and Welds (STP-NU-039)  
Creep-Fatigue Data and Evaluation Procedures for Grade 91 and Hastelloy XR (STP-NU-018)  
Evaluation of the NSQ-100 Nuclear Safety and Quality Management System Requirements (STP-NU-061)  
Extended Allowable Stress Values For Alloy 800H (STP-NU-035)  
Graphite for High Temperature Gas-Cooled Nuclear Reactors (STP-NU-009)  
Guide to Nuclear Crane Standards (STP-NU-015)

Improvement of ASME NH for Grade 91 Negligible Creep and Creep Fatigue (STP-NU-013)  
New Materials for ASME Subsection NH (STP-NU-042)  
Non Destructive Examination (NDE) and In-Service Inspection (ISI) Technology for High Temperature Reactors (STP-NU-044)  
Operating Condition Allowable Stress Values in ASME Section III Subsection NH (STP-NU-037)  
Regulatory Safety Issues in Structural Design Criteria of Section III-NH and for VHTR (STP-NU-010)  
Risk Initiatives in ASME Nuclear Codes and Standards (STP/NU-001)  
Roadmap for the Development of ASME Code Rules for Fusion Energy Devices (STP-NU-067)  
Roadmap to Develop ASME Code Rules for the Construction of High Temperature Gas Cooled Reactors (HTGRS) (STP-NU-45)  
Small Modular Reactor (SMR) Roadmap (STP-NU-072)  
Update and Improve Subsection N H – Alternative Simplified Creep-Fatigue Design Methods (STP-NU-041)  
Update and Improve Subsection NH – Simplified Elastic and Inelastic Design Analysis Methods (STP-NU-040)  
Verification of Allowable Stresses in ASME Section III-NH for Alloy 800H (STP-NU-020)  
Verification of Allowable Stresses in ASME Section III-NH for Grade 91 Steel (STP-NU-019)

#### Certification Programs

NDE Technician Personnel Certification  
N-Type Nuclear Component Manufacturer Certification  
QSC Material Supplier - Product Certification  
NQA Certification

Note: Other ASME standards may be applicable.

For more information visit <http://catalog.asme.org/> or call 1-800-THE-ASME (800-843-2763).