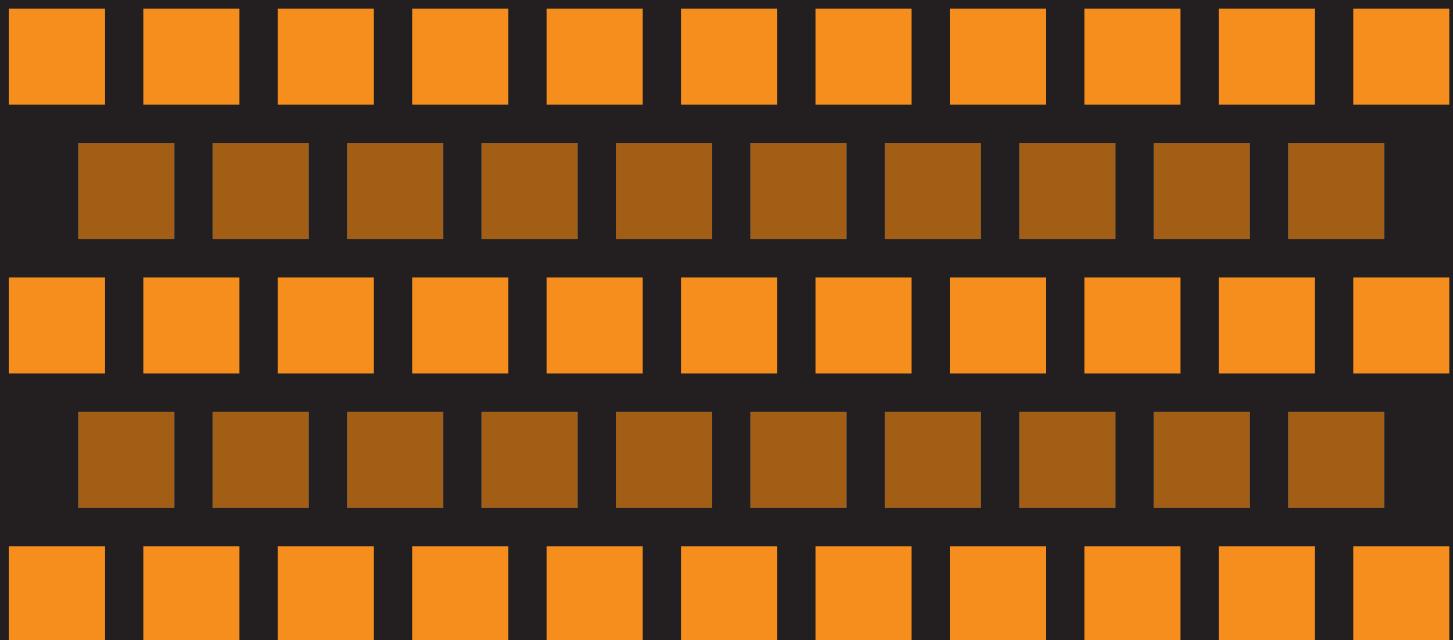


**STP-NU-013**

# **IMPROVEMENT OF ASME NH**

**FOR GRADE 91 NEGIGIBLE CREEP  
AND CREEP-FATIGUE**



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## FOREWORD

This document is the result of work resulting from Cooperative Agreement DE-FC07-05ID14712 between the US Department of Energy (DOE) and ASME Standards Technology, LLC (ASME ST-LLC) for the Generation IV (Gen IV) Reactor Materials Project. The objective of the project is to provide technical information necessary to update and expand appropriate ASME materials, construction, and design codes for application in future Gen IV nuclear reactor systems that operate at elevated temperatures. The scope of work is divided into specific areas that are tied to the Generation IV Reactors Integrated Materials Technology Program Plan.

ASME ST-LLC has introduced the results of the project into the ASME volunteer standards committees developing new code rules for Generation IV nuclear reactors. The project deliverables are expected to become vital references for the committees and serve as important technical bases for new rules. These new rules will be developed under ASME's voluntary consensus process, which requires balance of interest, openness, consensus, and due process. Through the course of the project ASME ST-LLC has involved key stakeholders from industry and government to help ensure that the technical direction of the research supports the anticipated codes and standards needs. This directed approach and early stakeholder involvement is expected to result in consensus building that will ultimately expedite the standards development process as well as commercialization of the technology.

ASME has been involved in nuclear codes and standards since 1956. The Society created Section III of the Boiler and Pressure Vessel Code, which addresses nuclear reactor technology, in 1963. ASME Standards promote safety, reliability, and component interchangeability in mechanical systems.

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## **ABSTRACT**

This report provides recommendations for improvement of ASME NH for Grade 91 in the areas of negligible creep and creep-fatigue.

The report is separated into the following four parts.

***Part I Improvement of ASME NH for Grade 91 (Negligible Creep)***

Examines the current approaches available to define negligible creep and checks their applicability to Grade 91 steel. The work is based on material data available in France and the U.S.

***Part II Improvement of ASME NH for Grade 91 (Creep-Fatigue)***

Compares Subsection NH and RCC-MR creep-fatigue procedures. Comparisons are performed on cases defined on the basis of experimental test results available from Japan, France and the U.S. on Grade 91 steel. Particular attention was paid to the definition of safety factors and creep-fatigue damage envelope. Improvements to existing procedures are recommended.

***Part III Proposed Test Program to Assess Negligible Creep Conditions of Modified 9Cr-1Mo***

Part III is aimed at defining tests necessary to validate negligible creep conditions for Mod 9Cr-1 Mo material.

***Part IV Proposed Test Program to Validate Creep-Fatigue Procedures for Modified 9Cr-1Mo***

Part IV completes the work performed in Part II which, on the basis of creep-fatigue tests results available from Japan, Europe and the US, compared creep-fatigue procedures of ASME Subsection NH and RCC-MR Subsection RB.